

Overview and Status of the National Ignition Campaign on the NIF

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NATIONAL IGNITION CAMPAIGN

















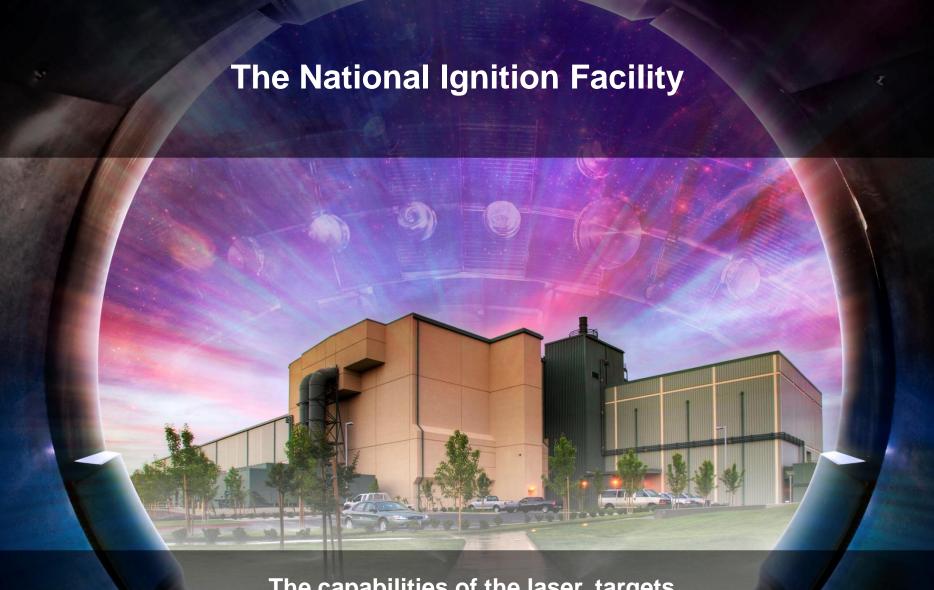




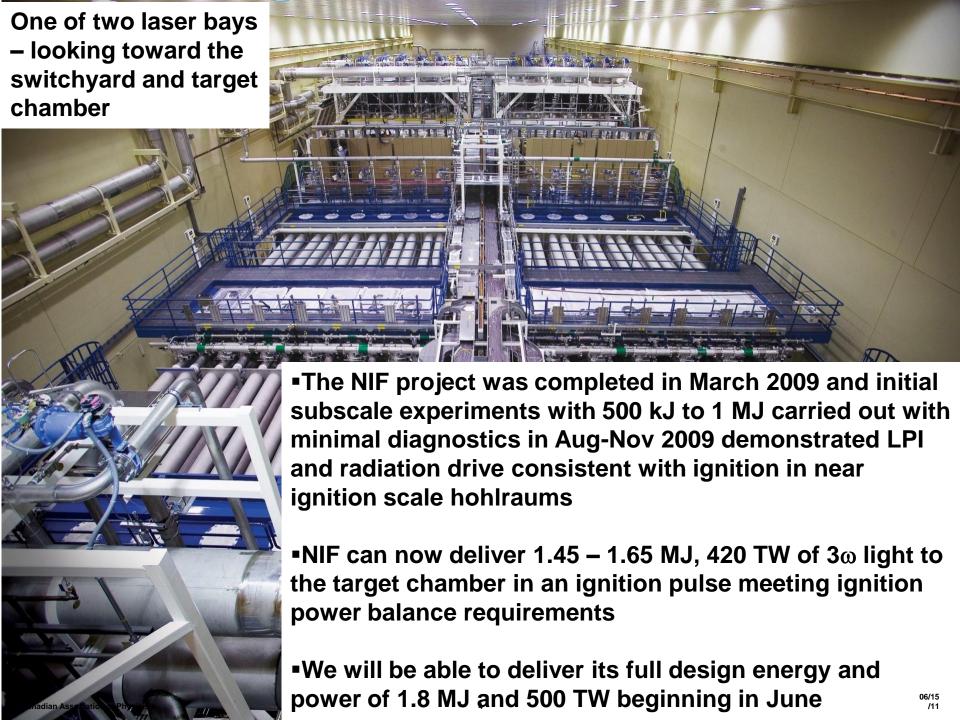








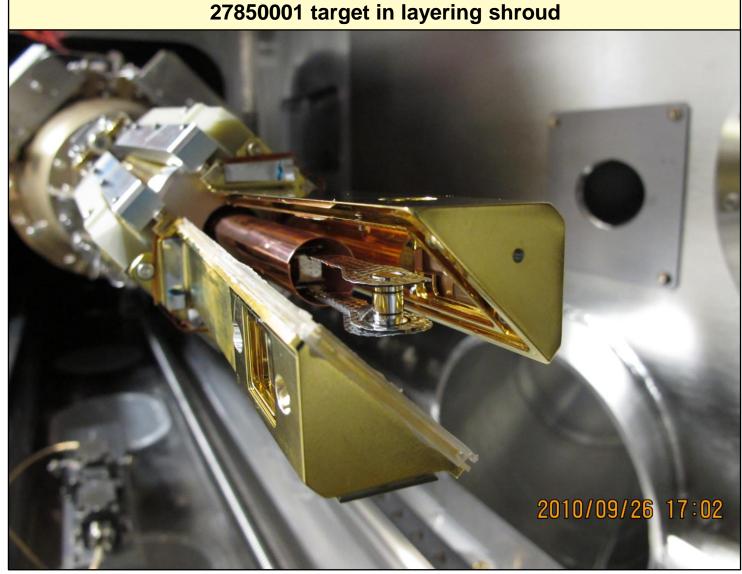
The capabilities of the laser, targets, diagnostics, and experimental platforms are now in place for the push to ignition





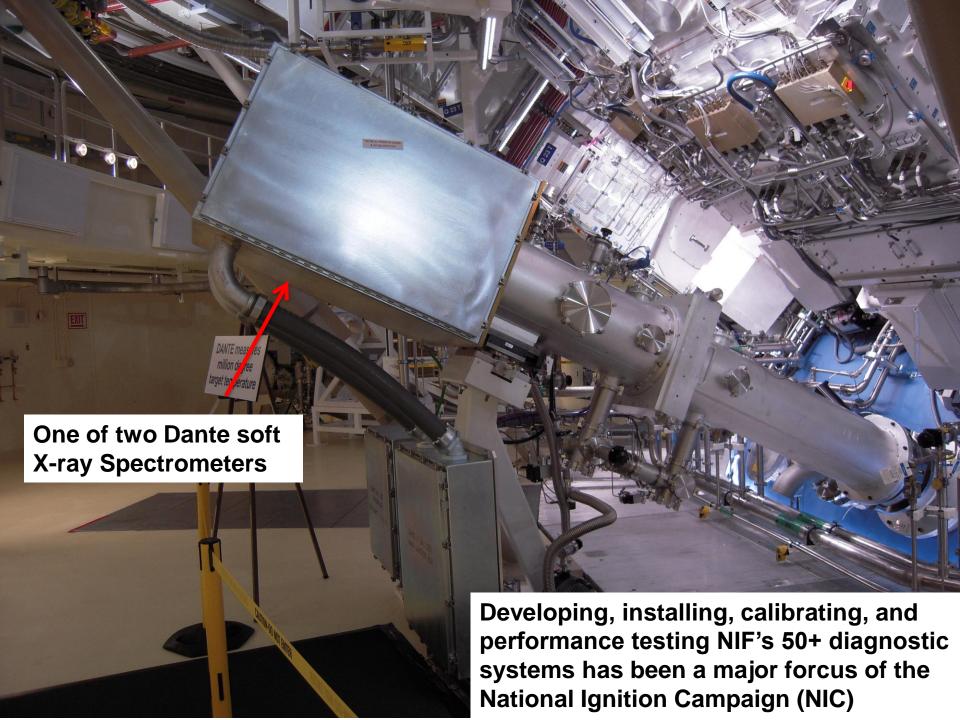
THD fuel layers are formed with the target mounted in a dedicated cryogenic target positioner thermally isolated by a removable shroud





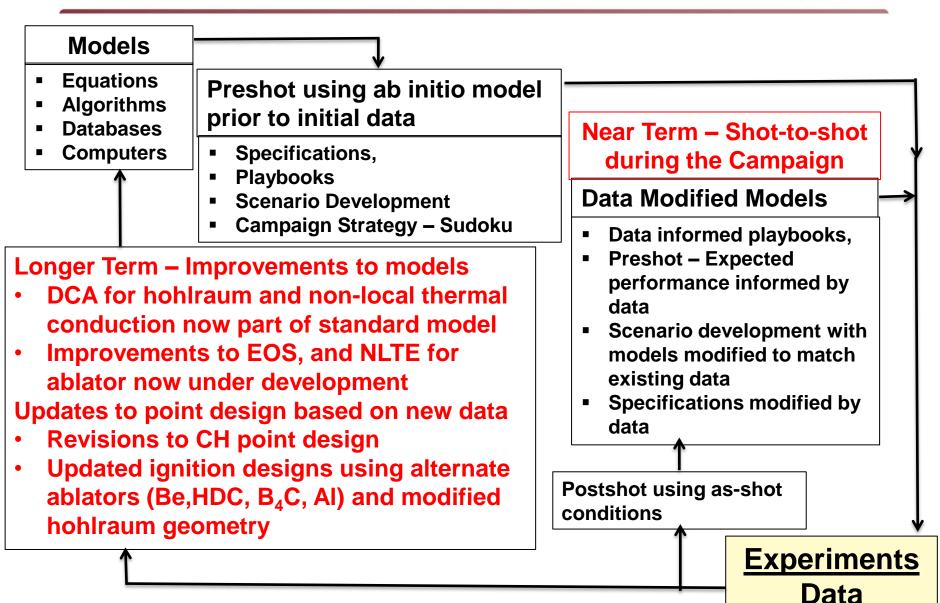
A multi-laboratory effort in fabrication has given NIF the production capability for targets with unprecedented precision







There are multiple time scales for the use and evolution of numerical models within the NIC



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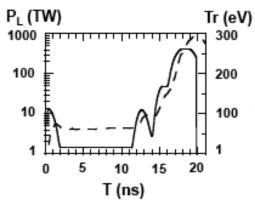


Summary of Ignition Campaign Status

- We are one year into the campaign to carry out precision optimization of ignition scale implosions
 - We have achieved hohlraum temperatures in excess of the 300 eV ignition goal with hot spot symmetry and shock timing near ignition specs
 - Slower rise to peak power and longer "no-coast" pulses result in lower hot spot adiabat and main fuel ρr at about 85% of the ignition goal
 - Nuclear data indicates that long wavelength variation in the main fuel density may be contributing to performance degradation
 - Mix performance boundary with more mass remaining than the point design will require thicker shells (+20-30%) to reach ignition velocity without mix



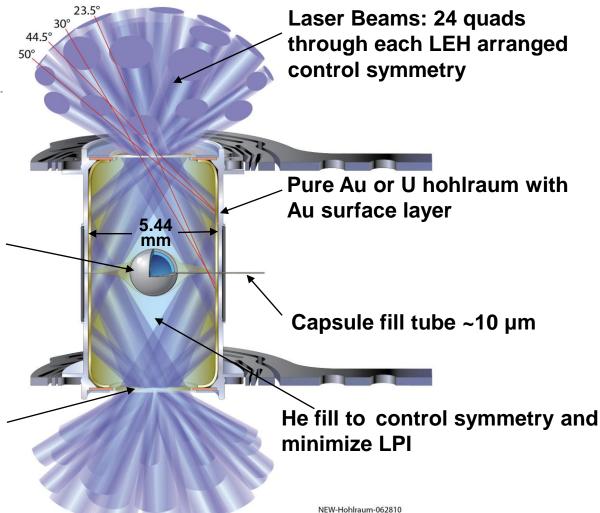
Ignition Target designs have a number of general features



Capsule with low-z ablator (CH, Be, or HDC*) and cryo fuel layer

Laser Entrance Hole sized to balance LPI and radiative losses – 56–60% of LEH diameter

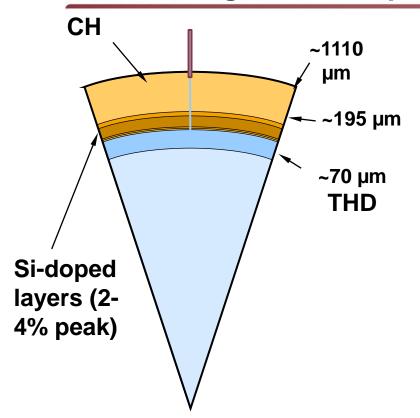
*High Density Carbon



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The initial ignition campaign is using a CH capsule

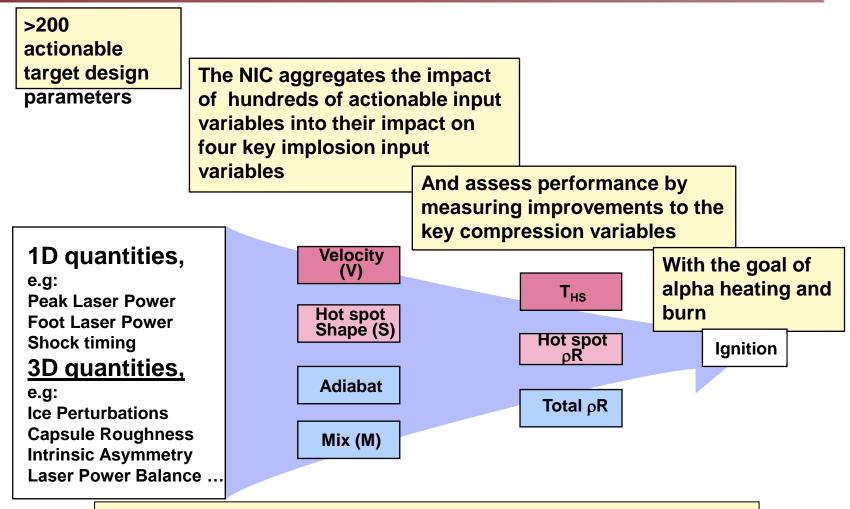


- Amorphous material with no crystal structure issues
- Large data base from the Nova and Omega (LLE) lasers
- Reduced Facility impact relative to Be
- All of the diagnostics and infrastructure needed for optimizing ignition implosions are essentially independent of capsule ablator

- Silicon doped layers reduce X-ray preheat at ablator-DT interface to make favorable Atwood number during acceleration to control mix
- Ablator thickness is adjusted to vary sensitivity to mix of fuel and ablator resulting from ablation front instability growth

To achieve ignition, the NIC must generate the data needed to optimize the principal characteristics of an ICF implosion

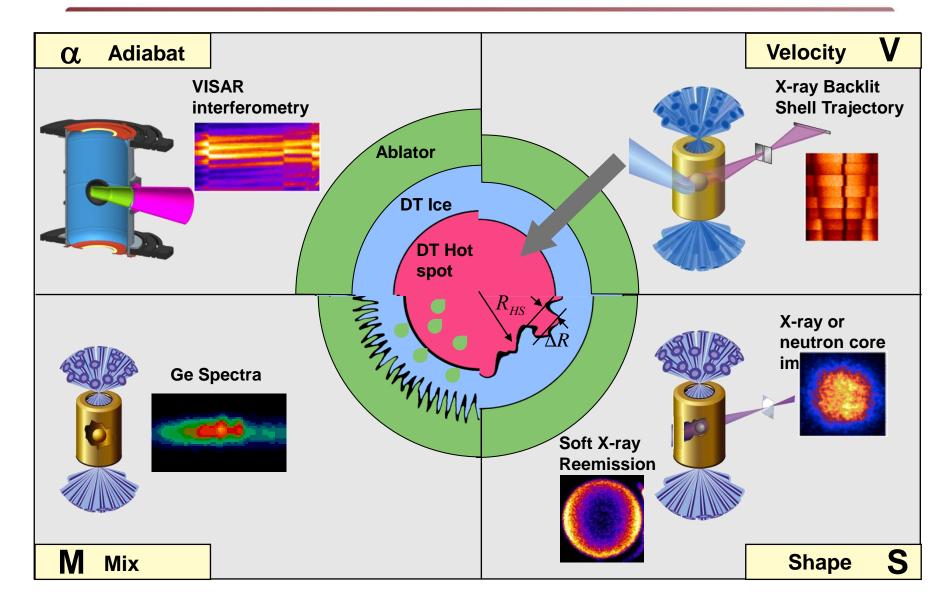




- The key variables for ICF have been known for decades
- Since NIF was first proposed, we have worked to better quantify the specifications for ignition at the megajoule scale

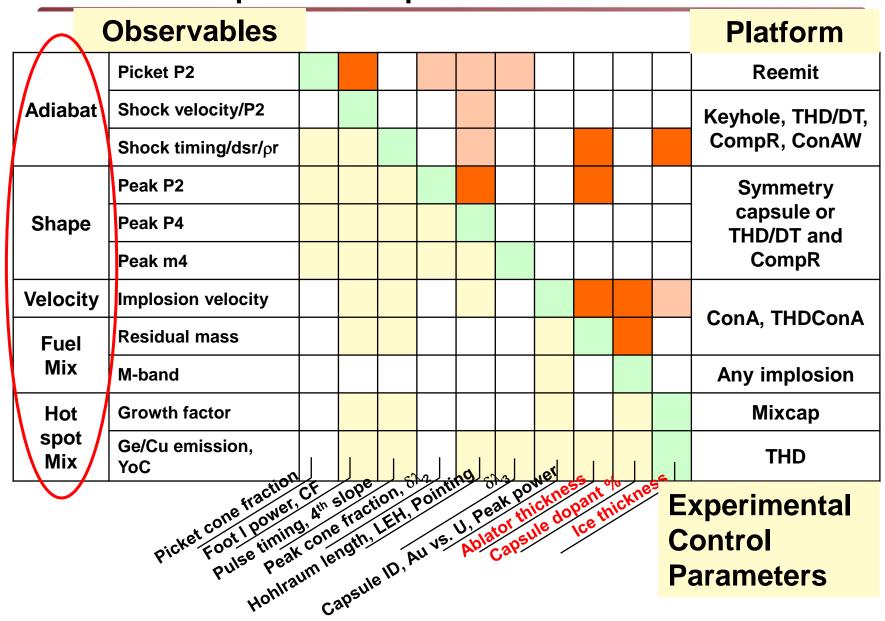
From September 2010 to April 2011, the NIC focused on validating a series of experimental platforms to optimize the capsule shape, adiabat, velocity, and mix





The National Ignition Campaign (NIC) is designed to generate the data needed for an optimal implosion in the most efficient sequence of experiments





We began precision optimization experiments in May 2011 and completed the first pass through all key variables in April 2012 with the first mix campaign

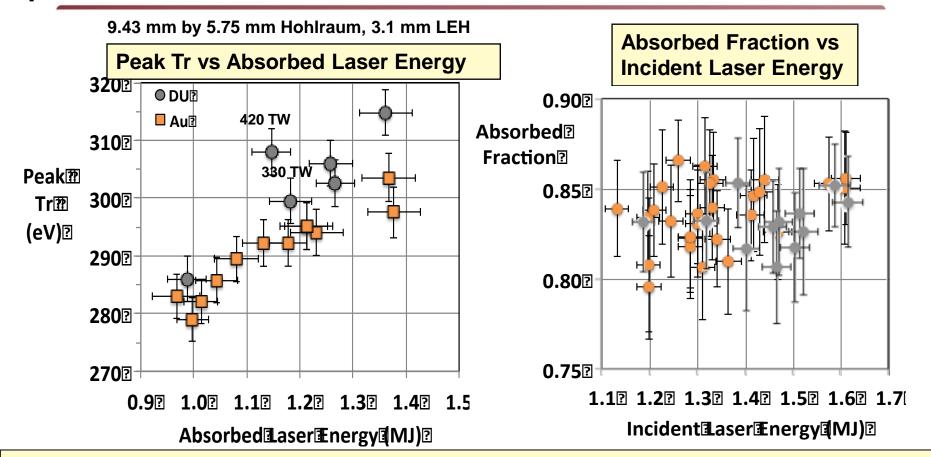


Observables					,	May 2011 - first					Platform		
	Picket P2			K		precision shock timing experiments						Reemit	
Adiabat	Shock velocity/P2									ner	its	Keyhole, THD/DT,	
	Shock timing/dsr/pr			1								CR, ConAWide	
Shape	Peak P2				>							Symmetry	
	Peak P4					V						capsule or	
	Peak m4						V			\		THD/DT and CR	
Velocity	Implosion Velocity							VV				ConA, THDConA	
Fuel Mix	Residual mass											·	
	M-band	N	March/April 2012– first iteration on mix optimization								Any implosion		
Hotspot	Growth factor										Міхсар		

- The optimal sequencing was studied extensively prior to the start of experiments in the Red Team / Blue Team study
- That sequencing has been largely validated in experiments although a few new experimental platforms have been added to those originally envisioned and others may be needed to achieve ignition

We have achieved the ignition goal of Tr > 300 eV with coupling of 83 2% nearly independent of laser energy up to 1.6 MJ

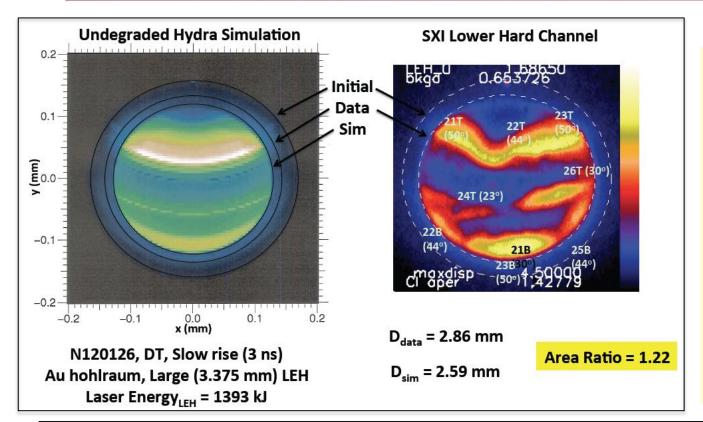




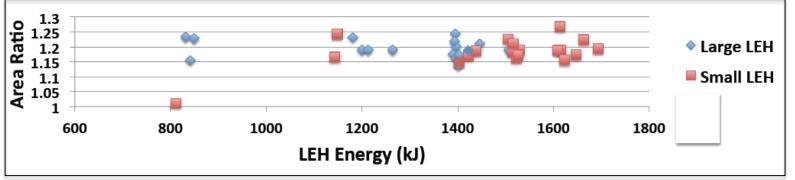
- 17% LPI losses are about twice what was anticipated prior to first experiments
- Increased loss is consistent with improved understanding of the plasma conditions resulting from the implementation of the DCA NLTE atomic physics model and non-local electron transport which results in increased importance of multi-guad overlap effects on LPI



Standard calculations overestimate laser entrance hole closure by about 20%

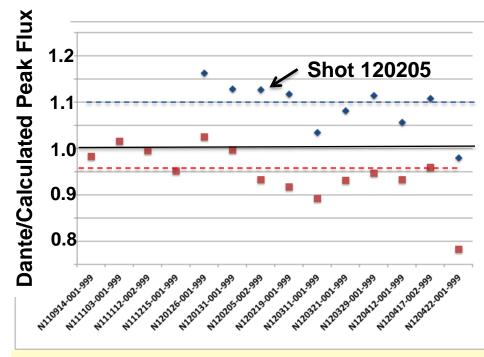


- Recent zoning studies indicate that much of this discrepancy could be numerical
- Heating of the blowoff plasma by various plasma processes not included currently could help keep LEH open



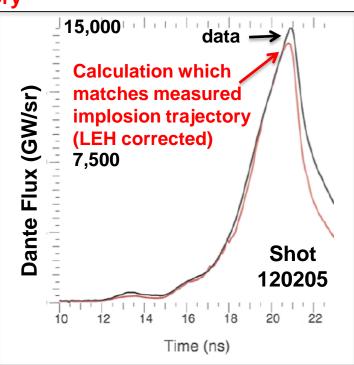
When corrected for laser entrance hole size, calculations with the flux versus time needed to match implosion trajectories, match the Dante peak flux to about 4%





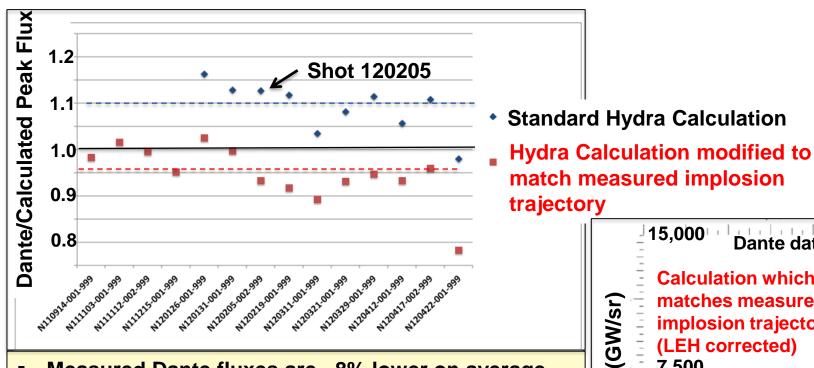
- Standard Hydra Calculation
- Hydra Calculation modified to match measured implosion trajectory

- Standard calculations overestimate the measured Dante flux by about 8% on average
 - much of this difference may be explained by numerical zoning effects in calculations
- Observed shell trajectories are consistent with about 4% less flux than observed on average
 - NLTE effects in the ablator are predicted but currently estimated to be small

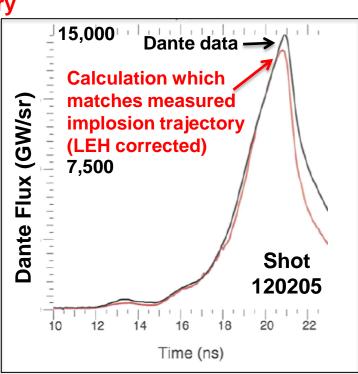


Measured fluxes corrected for the observed LEH closure provide the best estimate when comparing data to calculations of the hohlraum drive and capsule response



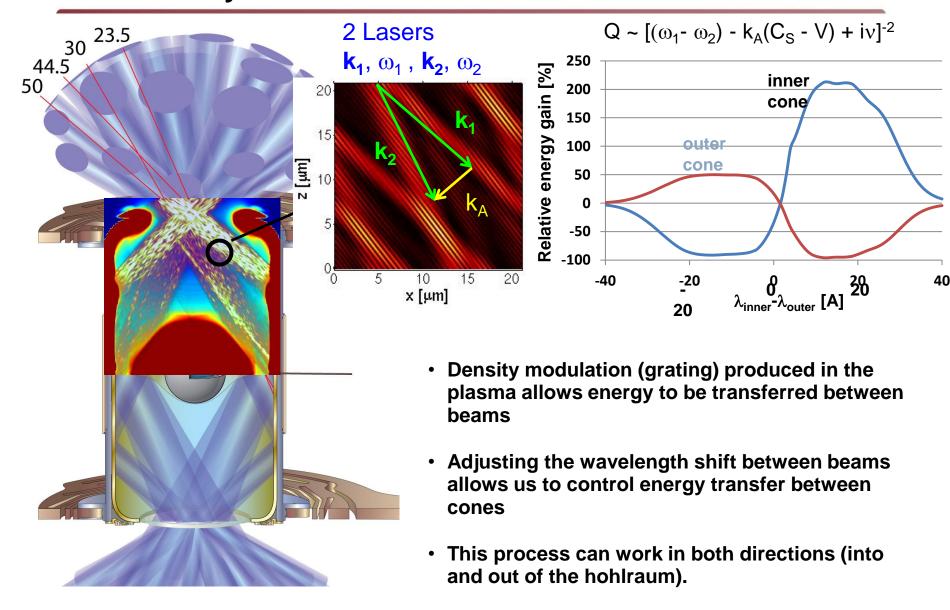


- Measured Dante fluxes are ~8% lower on average than standard Hydra calculations
 - much of this difference may be explained by numerical zoning effects in calculations
- Observed shell trajectories are responding as if the flux were about 4% less than Dante on average
 - NLTE effects in the ablator are predicted but currently estimated to be small



Crossed laser beams in the hohlraum plasmas produce intensity modulations that drive density modulations





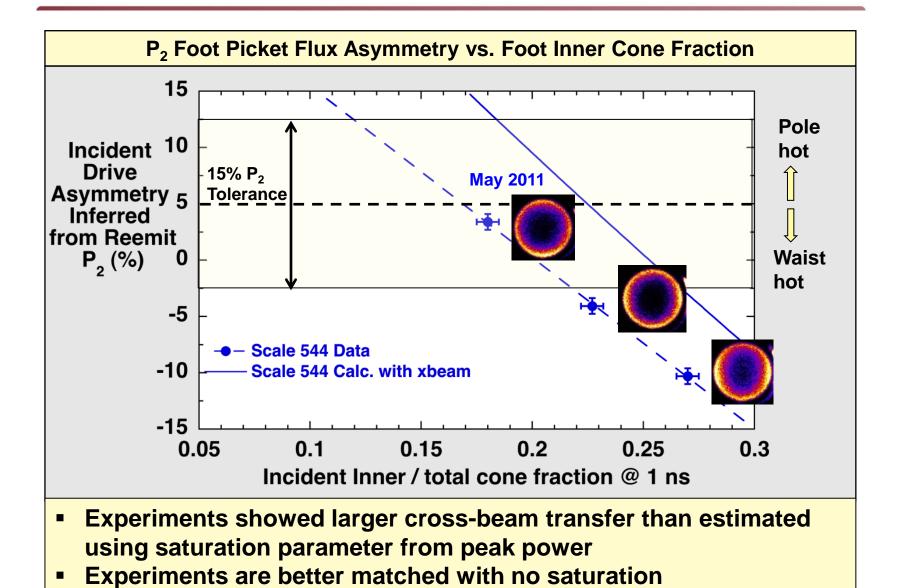


Reemit Target sets the cone power ratio for the first 2 ns to ensure symmetric foot drive

Experimental Geometry Bi sphere "Reemit" replaces layered capsule Nov. 2010 0.7 keV X-ray images 2 mm Observable: Limb brightness vs angle

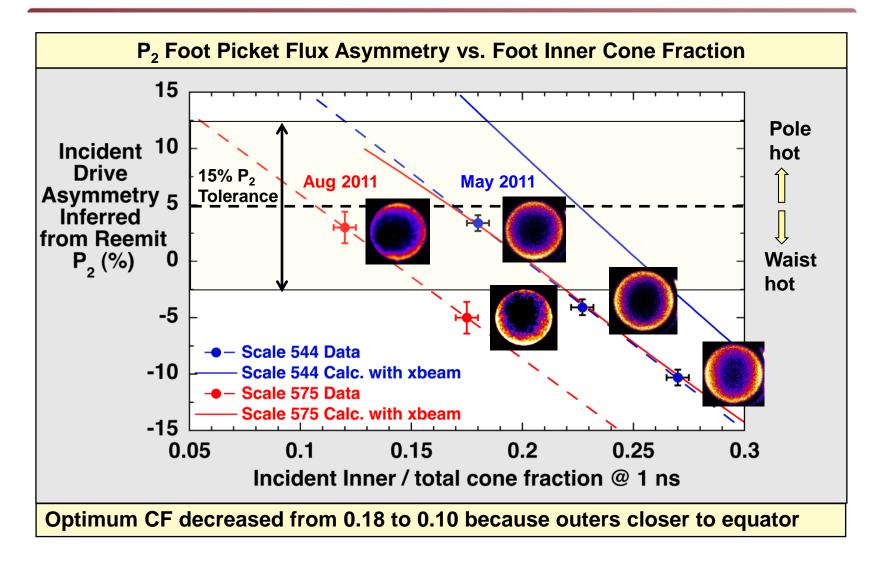


May 2011: Re-emit brightness set picket cone fraction to 1% in the Scale 544 hohlraums used before August 2011





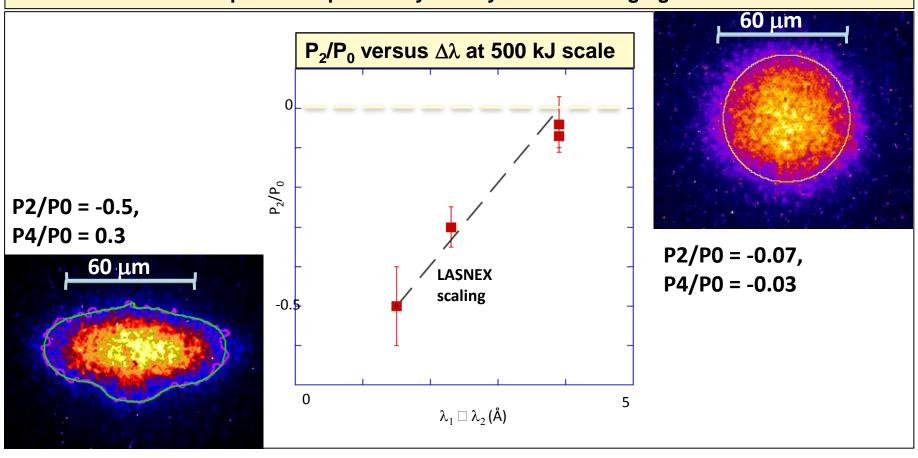
Aug 2011: Re-emit confirmed expected lower inner cone fraction for Scale 575 hohlraum





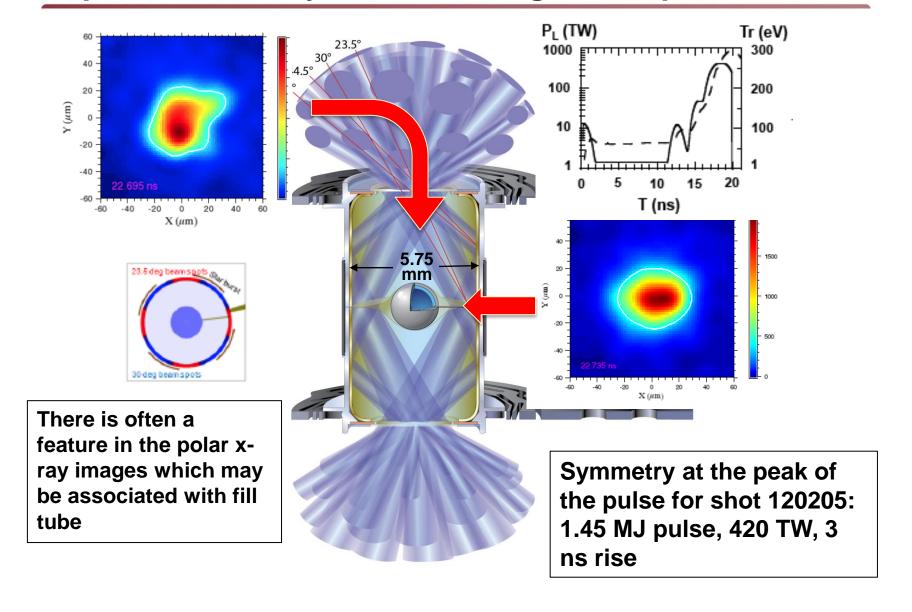
Implosion symmetry at the peak of the laser pulse is achieved by tuning the wavelength of the outer cone

First demonstrated in experiments at 500 kJ in 2009, tuning the $\Delta\lambda$ between inner and outer beams allows us to optimize implosion symmetry without changing the laser cone fraction



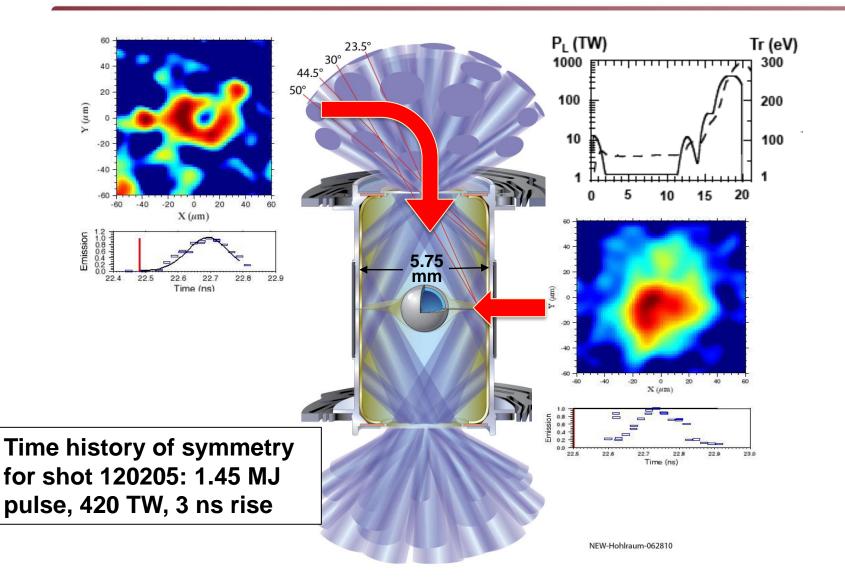


Hot spot symmetry demonstrated in recent implosions is very close to the ignition specs





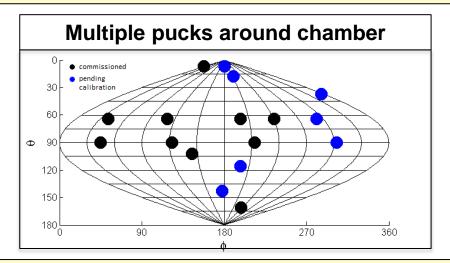
Hot spot symmetry demonstrated in recent implosions is very close to the ignition specs



Nuclear measurements indicate that the main fuel can have large ρR variations even when the hot spot appears quite symmetric

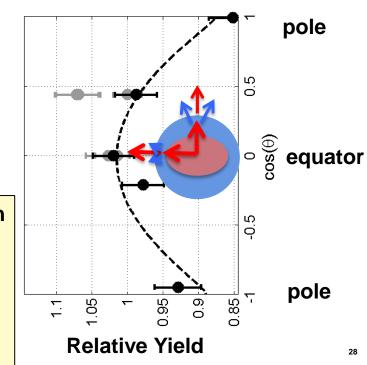


FNADS (Flange Nuclear Activation Detectors) are Zirconium threshold detectors which measure the primary neutron yield



- Some shots show significant signal variations (high ρR on poles) on a typical shot with ρR~1 g/cm² in DT, about 20% of the neutrons are downscattered, so a 10% variation in the measured primary yield corresponds to a 50% ρR variation (needs better calibration for required accuracy)
- ρR variations also indicated by Neutron Time of Flight (NTOF) and Magnetic Recoil Spectrometer (MRS) data

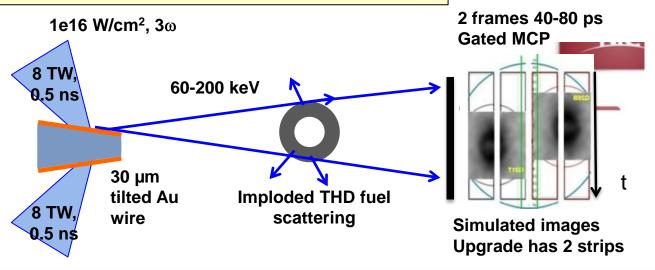
Count activation and analyze (sometimes large asymmetry)

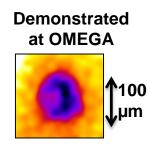


We are developing imaging diagnostics which will give us improved shell-in-flight and compressed fuel measurements

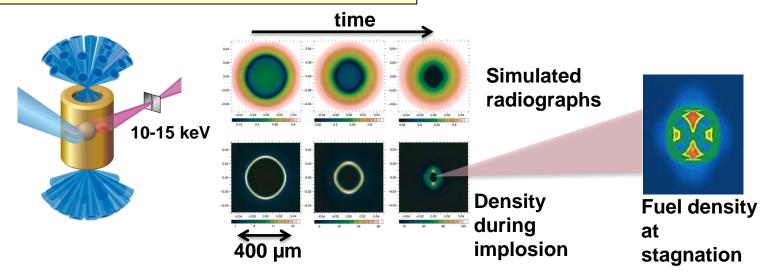


Compton radiography – DT fuel at stagnation



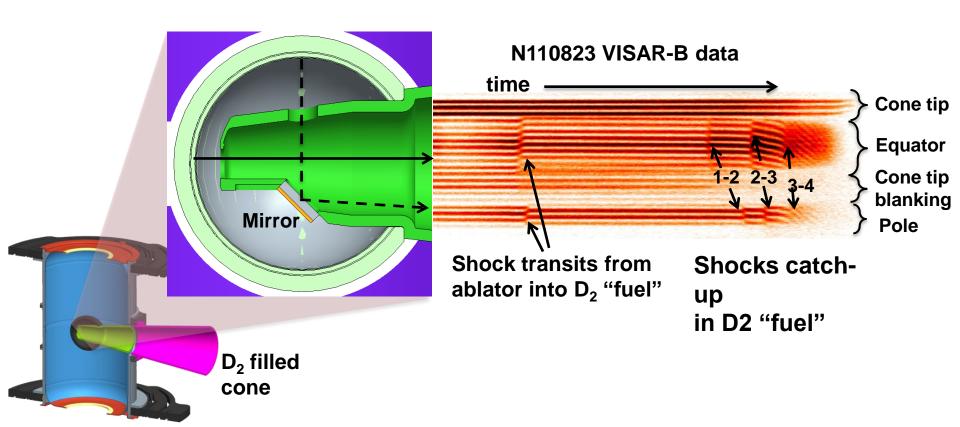


2D absorption radiography – shell in-flight



The mirrored keyhole targets are used to optimize shock timing and velocity as well as the pole to waist asymmetry for all 4 shocks in the pulse



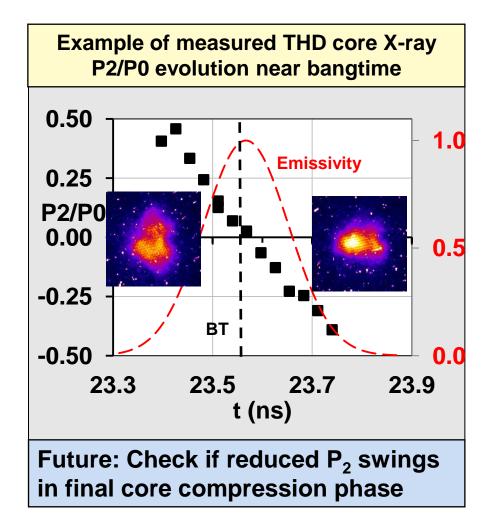


Calculations show that asymmetric 2nd and 3rd shocks give rise to symmetry swings in the imploded core

THD

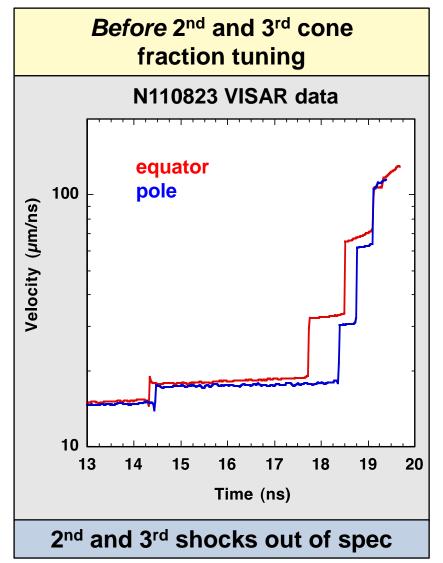


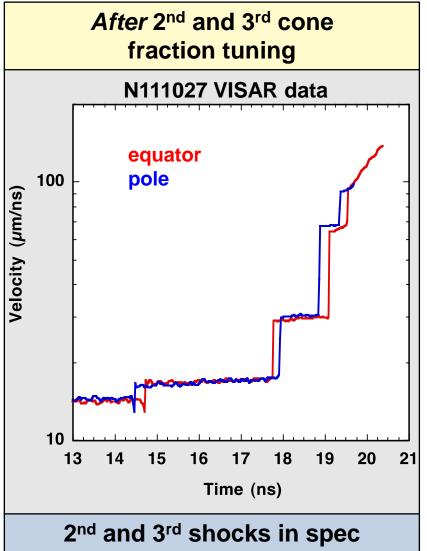
Shock asymmetries can lead to P_2 swings in core shape and fuel ρr nonuniformities





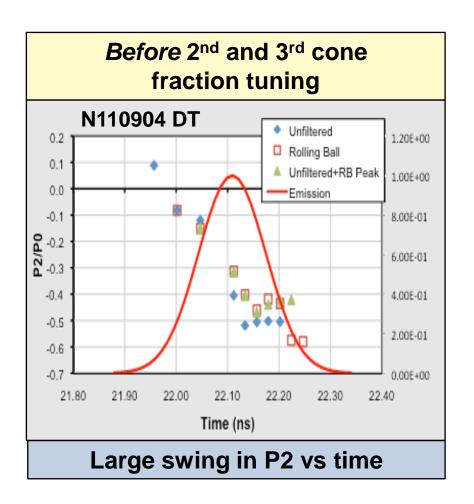
Mirrored keyhole experiments were used to improve the shock symmetry

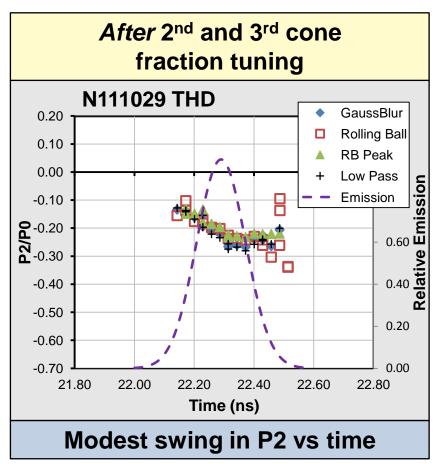






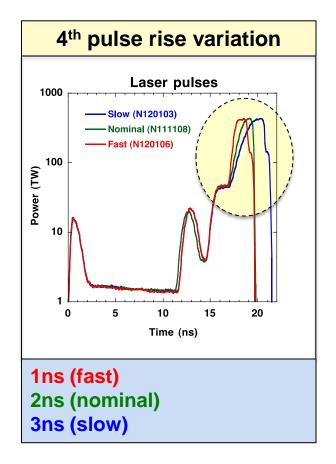
Swings in symmetry are reduced after 2nd and 3rd cone fraction optimization

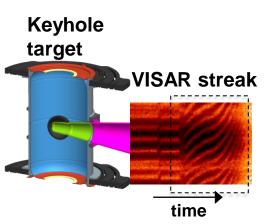


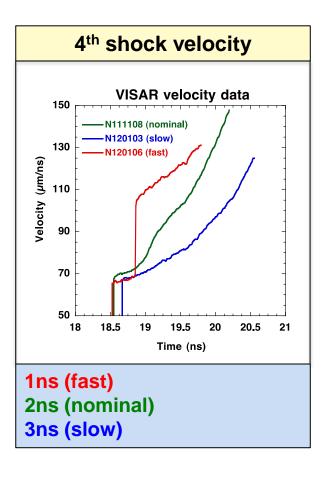


The first tests of the impact of variations in the temporal shape of the peak power pulse changed the rate of rise to peak power





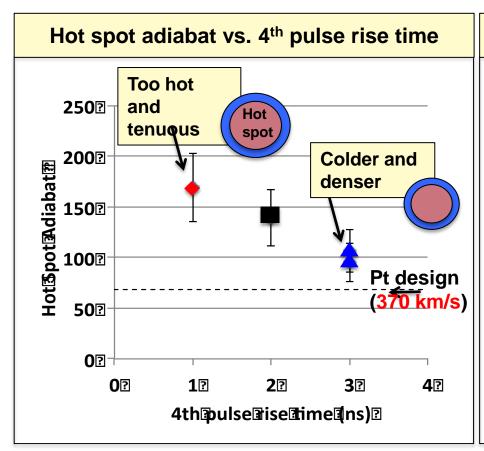


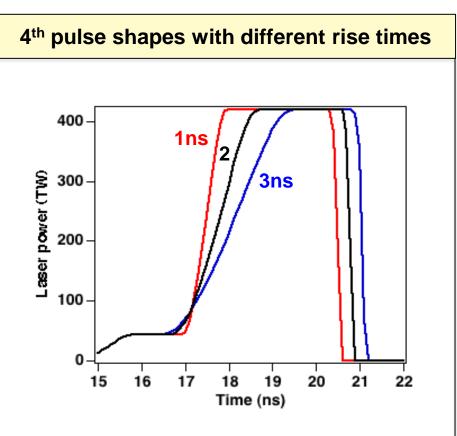


Slower rise pulses are predicted to be less sensitive to fluctuations in drive in 4th rise

NIC

Slower rise 4th pulses have produced hot spot adiabats closer to ignition goals



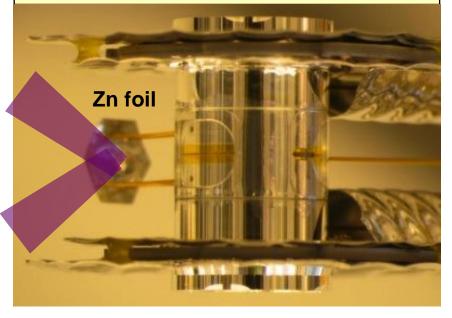


As a result of these tests, we adopted the slower "3ns rate of rise" as the primary pulse shape through April 2012

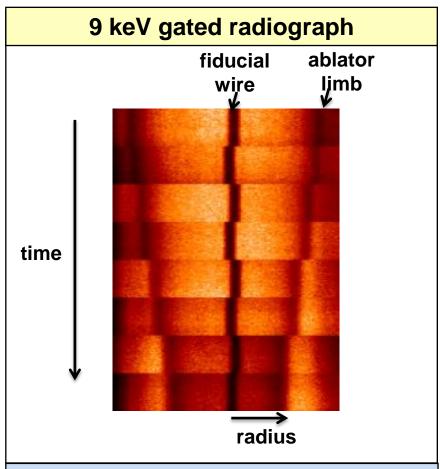


Backlit Capsule sets peak power and capsule ablator thickness (trading off velocity vs mix susceptibility)

Capsule backlit by x-rays from separate laser plasma



Until March 2012, implosion kinematics were measured using gated backlit radiography

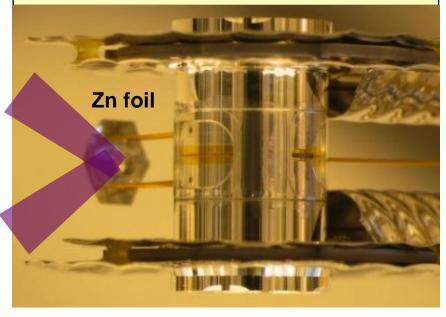


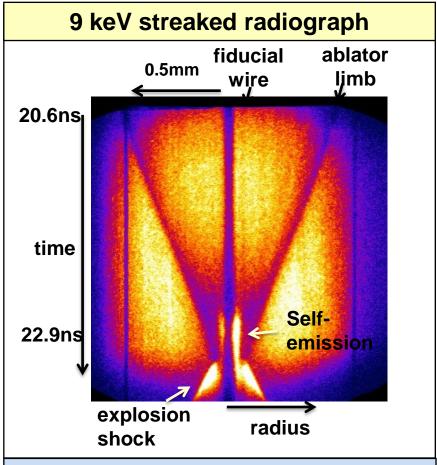
Technique measures shell radius, velocity, ρR profile, and remaining ablator mass



In March, we activated streaked backlit radiography (3/24/12)

Capsule backlit by x-rays from separate laser plasma



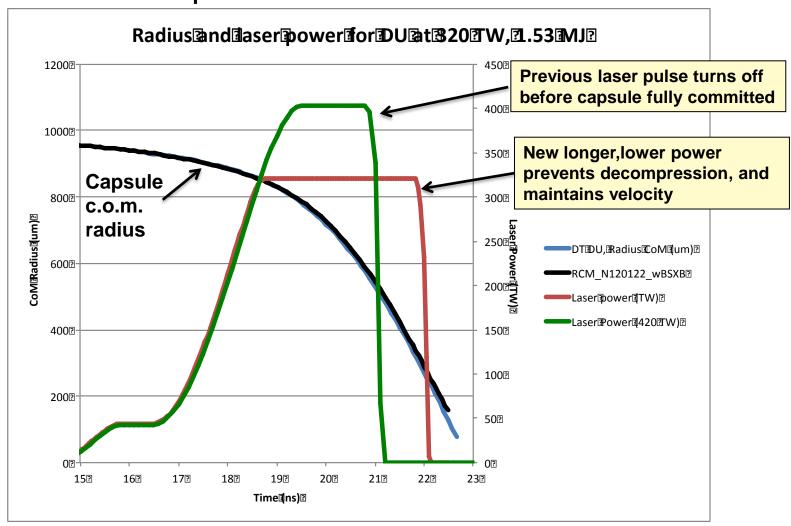


Provides continuous record of ablator kinematics



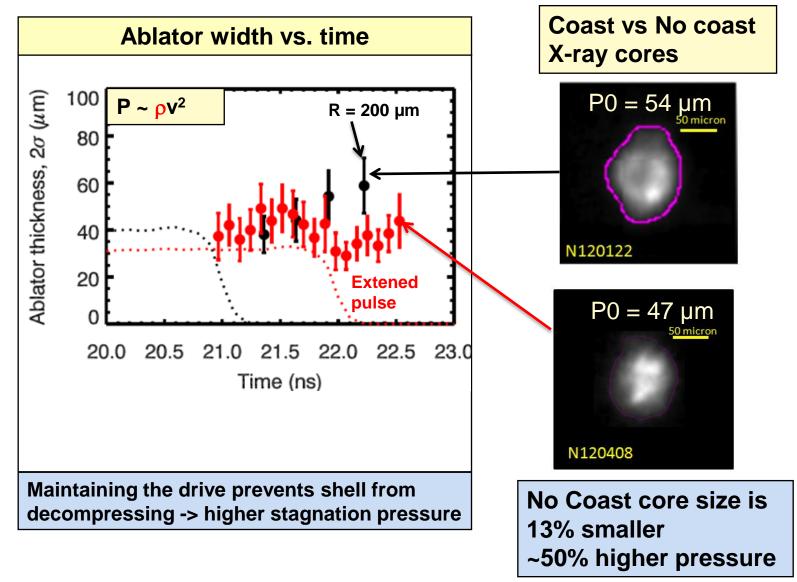
In March, experiments moved to longer "no-coast" pulse to avoid capsule decompression prior to stagnation

Simulated capsule center of mass radius vs time





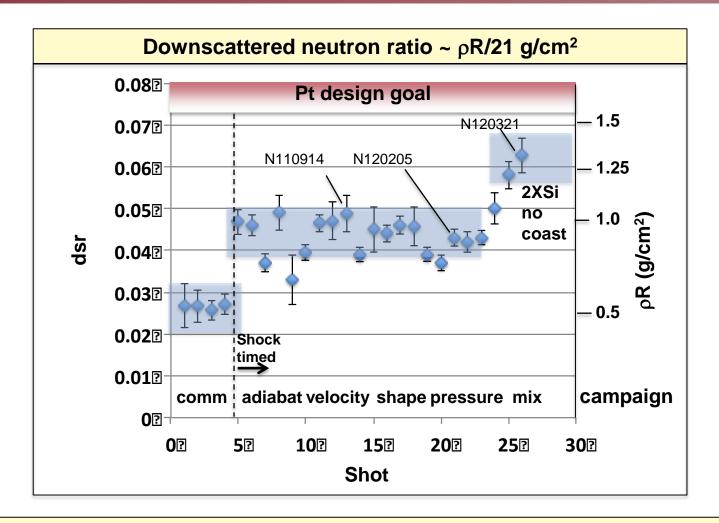
"No Coast" behavior: Ablator stays compressed by extending pulse out to $r = 300 \mu m$



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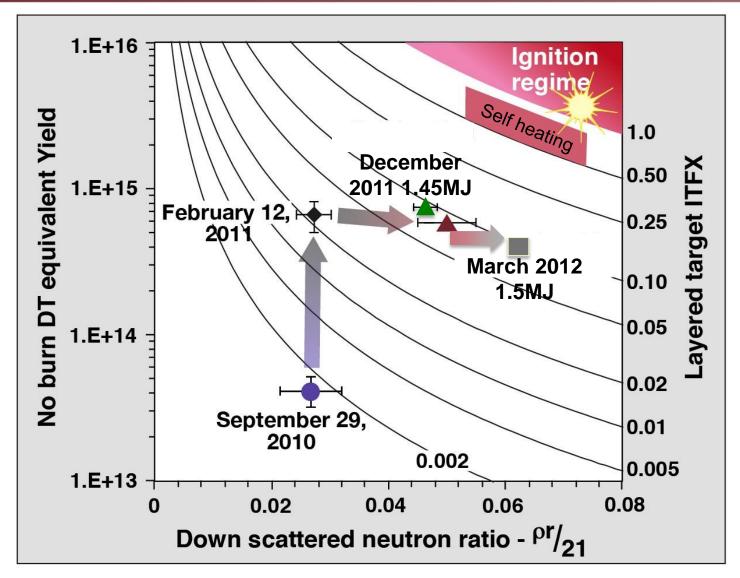
Feb-March 2012 Campaigns increased down scattered neutron ratio (dsr) ~ ρR / 20



Recent improvement attributed to reduction in coasting (longer laser pulse) and reduction in interface mix (2XSi dopant in ablator reducing preheat)

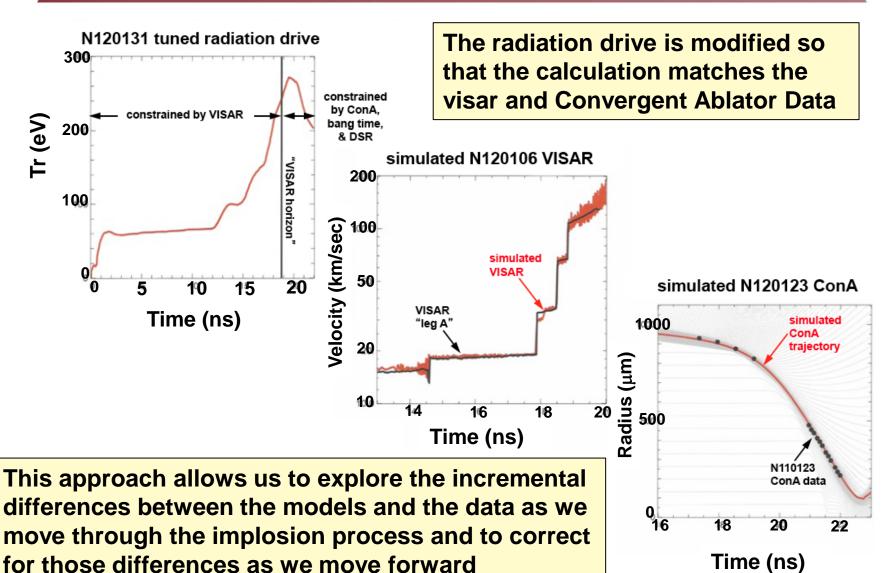
Fuel pr is now at about 85% of the ignition point design but we need to increase yields a factor of 5-10 to get into the strongly self-heated regime

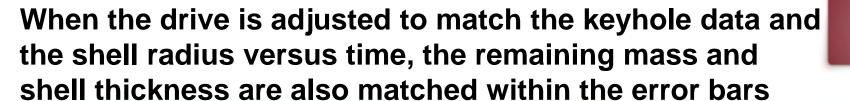




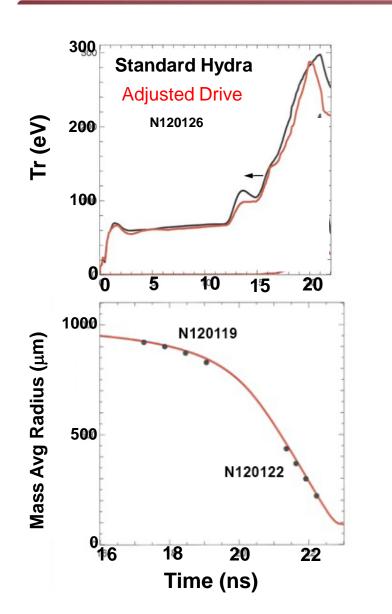
We have developed a standardized approach for generating 1D capsule drives used in calculating cryo-layered capsule performance

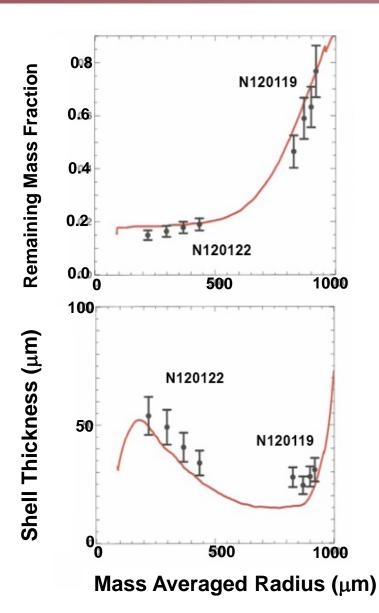






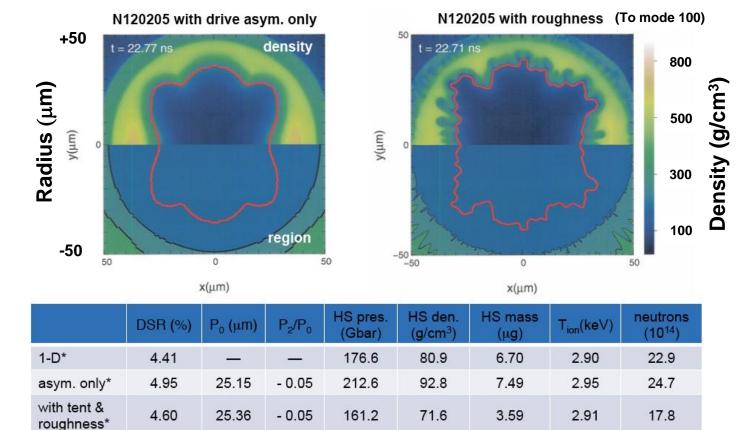






Calculations of layered implosions with these modified drives match much of the observed data but typically over estimate yields by a factor of several





22.89

4.54

N120205

 Calculations do not include the known 3D long wavelength asymmetry in the capsule, hohlraum, and laser power

105.1

44.3

4.80

3.39

5.64

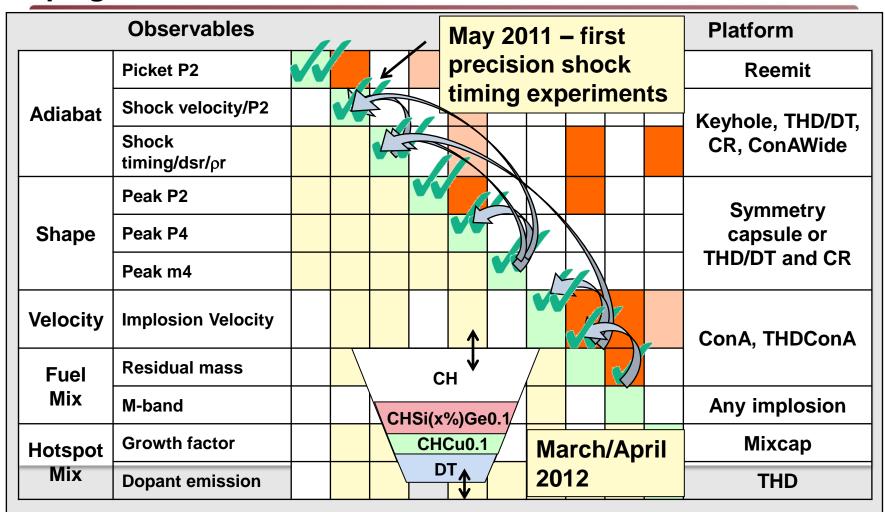
3D calculations to mode 100 are under development

-0.15

^{*} with α particle momentum and energy deposition switched off



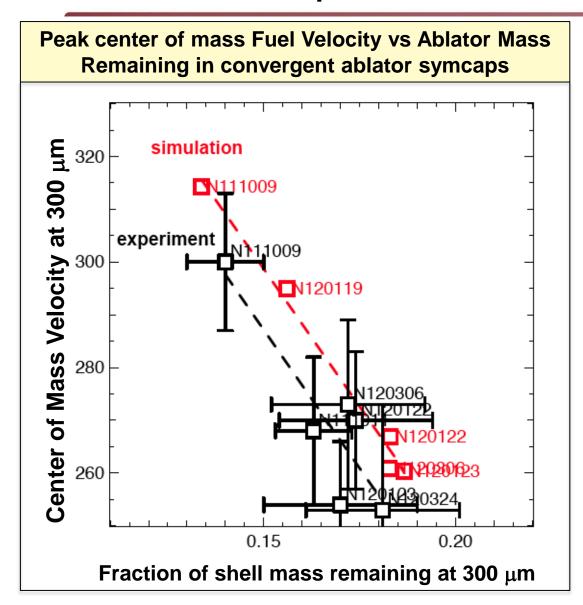
We have just completed the first iteration on a mix campaign



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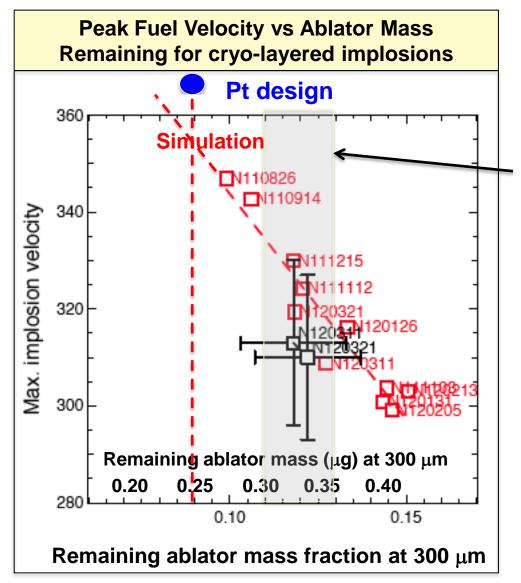
CH Ablation driven implosions follow a rocket curve which allows us to explore mix versus velocity



- Capsule drive in the simulations are adjusted to match the keyhole shock timing data and the convergent ablator radius versus time
- Data is estimated to have 1% less mass remaining at a given velocity than the 1D simulations
- We are exploring whether hydro instability in the imploding shell contributes to this difference

Calculations of the convergent ablator experiments are used to assess the velocity and remaining mass in cryo layered implosions

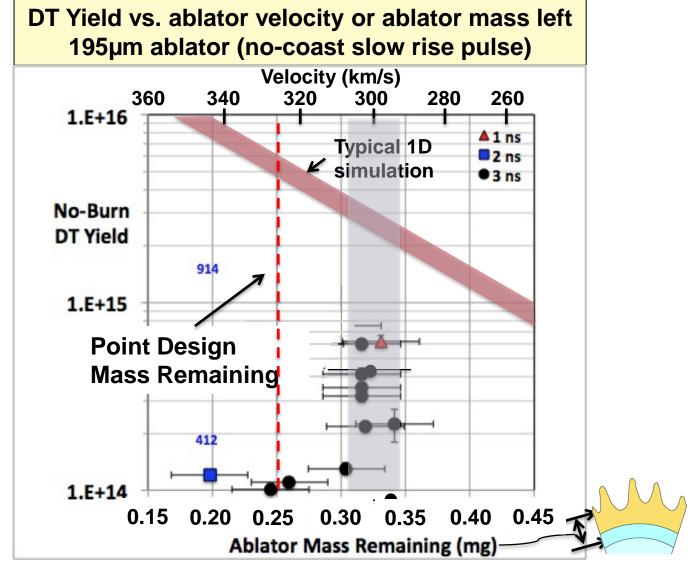


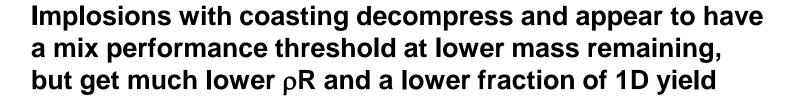


 We find a mix performance boundary at 20-40% more mass remaining than that calculated for the point design

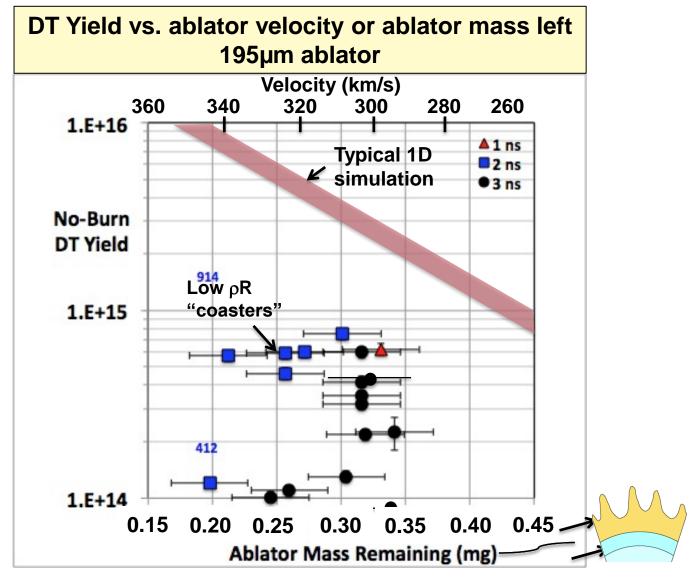
We find a fairly sharp performance boundary with ~20-40% more ablator mass remaining than that for the point design





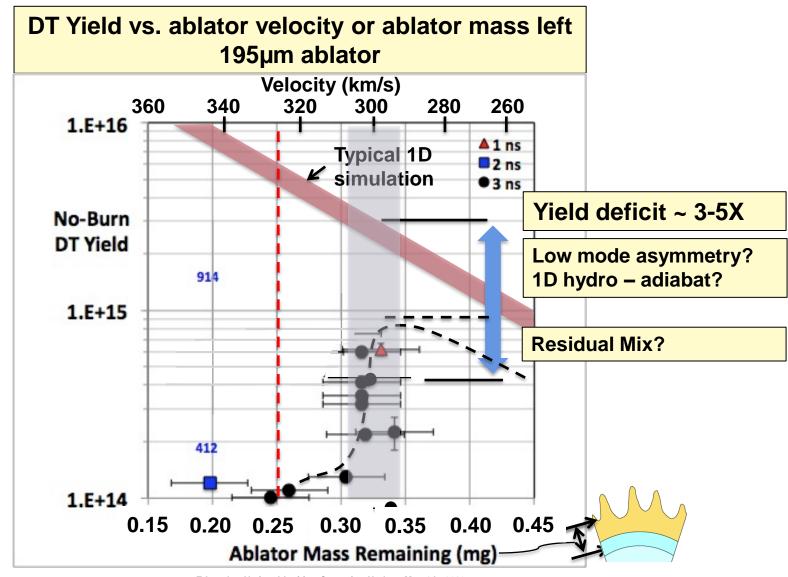






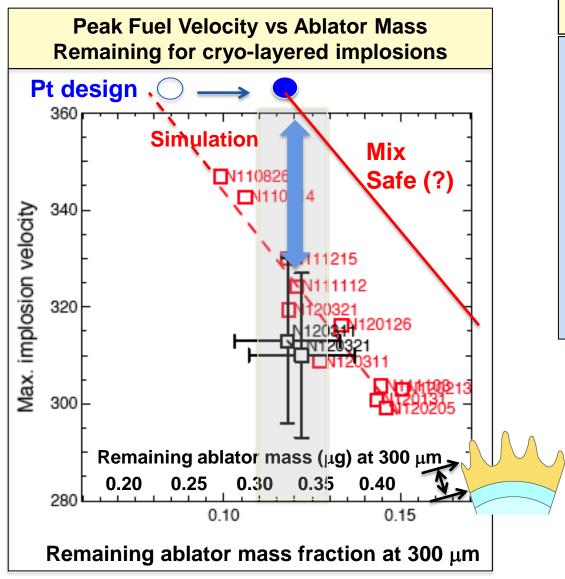


Increasing the yield a factor of 3-5 yield at the current velocity is a key element of upcoming experiments





To get to the point design velocity, we need to increase velocity while keeping mass remaining "mix safe"



 $V_{imp} \sim \sqrt{(ZTr/A)In(M_0/M_r)}$

Per Rocket Model: +20% V_{imp}, same M_r:

+20-30% M₀ (+40 to 60 μm) +10% Tr (290 to 320 eV)

Same DU hohlraum: +40-50% Peak Power (320 to 450-500 TW) +0.4-0.5 MJ (1.9 to 2 MJ)

- Improve capsules to reduce seeds
- Measure RT and RM growth to identify ways to reduce growth
- Reduce low mode asymmetry to minimize "thin spots" in fuel



Summary of Ignition Campaign Status

- We are one year into the campaign to carry out precision optimization of ignition scale implosions
 - We have achieved hohlraum temperatures in excess of the 300 eV ignition goal with hot spot symmetry and shock timing near ignition specs
 - Slower rise to peak power and longer "no-coast" pulses result in lower hot spot adiabat and main fuel ρr at about 85% of the ignition goal

These areas plus the temporal history of the main pulse will be the focus of ignition experiments moving forward

- Nuclear data indicates that long wavelength variation in the main fuel density may be contributing to performance degradation
 - Mix performance boundary with more mass remaining than the point design will require thicker shells (+20-30%) to reach ignition velocity without mix



